

### **Control integration**

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11th ITER International School San Diego, CA, USA 25-29 July 2022

SWISS PLASMA CENTER



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### **TCV: Tokamak à Configuration Variable**



B<sub>t</sub> = 1.5T, I<sub>p</sub><1MA, a=0.25m R=0.88m  $\kappa$ <2.8 19 separately controllable PF coils



### **TCV: Tokamak à Configuration Variable**



- Shape effects on plasma
  - Core, edge studies
  - Negative triangularity, snowflakes
- NBI and ECRH heating
- Plasma control research
  - Formation of advanced plasma shapes & vertical control (1990s-...)
  - MHD control: NTMs, Sawteeth (2000-...)
  - State estimation & Profile control (2010-...)
  - Integrated control (2015-...)

+ Flexible digital control system (100% Simulink-programmable)

B<sub>t</sub> = 1.5T, I<sub>p</sub><1MA, a=0.25m R=0.88m  $\kappa$ <2.8 19 separately controllable PF coils



Part 1: Integrated control: architectures and some examples of solutions

Part 2: Software engineering aspects of plasma control integration



#### Integrated control: key issues and some examples of solutions

### Motivation: future tokamak reactors will need to fulfil multiple control tasks with a limited set of actuators

- New control challenges:
  - Simultaneous execution of several (complex) control tasks with scarce actuators.
  - Real-time prioritisation of these tasks based on evolving plasma state/events.
  - Real-time automated assignment of scarce actuators to fulfil various tasks.



### Traditional control architectures with separate controllers are not sufficient for next-generation tokamaks





### Traditional control architectures with separate controllers are not sufficient for next-generation tokamaks



- Issues for integrated control:
  - Interaction/competition between controllers
  - Time-varying priorities for control
  - Time-varying actuator availability
  - Response to off-normal events

#### **Control integration via Multivariable Controller Design**

- Multivariable (MIMO) controller design
  - Design one controller that takes interactions into account explicitly.
  - Necessary when problems are strongly coupled dynamically.
  - Quickly becomes intractable as size of system increases.
  - Examples:
    - Shape control (many coils -> many shape control parameters) [DeTommasi lecture, Tue]
    - q profile (+betaN) control (many control points -> several actuators) [Schuster lecture, Wed]



But: we can not (yet) make one single controller for everything - we will have several separate controllers

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#### **ITER PCS** architecture design:

#### Supervision layer, controllers, support functions





### Supervisory control architectures under study in existing tokamaks

#### • DIII-D / KSTAR / EAST:

- Finite state Off Normal Fault Response (ONFR) [1]
- ASDEX-Upgrade / ITER:
  - Local/Global exception handling [2],
- TCV:
  - Supervision Actuator Management and Off-Normal Event handling (SAMONE) [3]
    - Control 'task' based approach, described in more details next

[1] N. W. Eidietis, et al, Nucl. Fusion, vol. 58, no. 5, p. 056023, May (2018).[2] W. Treutterer et al, Fus. Eng Des. 117, (2017)[3] Vu IEEE TNS (2021) and references therein

#### Control tasks:

- Tokamak independent, general formulation for any tokamak
- Represents 'something' that needs to be done by the control system

#### Separate responsibilities for task handling:

- A <u>supervisor</u> decides control task priorities based on plasma state.
- A set of <u>controllers</u> execute one or more control tasks: receiving plasma state information and compute actuator requests
- An <u>actuator manager</u> decides allocation of resources for prioritized control tasks

#### **Examples of control tasks:**

- 3/1 NTM preemption
- 2/1 NTM stabilization
- track q profile reference
- track β reference
- track Ip reference
- track V<sub>loop</sub> reference
- go to H mode
- stay in H mode

















### Architecture of task-based PCS: separation between specific interface layer and generic task layer





### Architecture of task-based PCS: separation between specific interface layer and generic task layer





### Plasma state reconstruction: combine specific diagnostic signals into to generic tokamak state descriptions



[1] C. Galperti et al., IEEE Trans. Nucl. Science 64 (2017) 1446-1454
[2] F. Felici et al., 26th IAEA FEC, 2016 [3] J-M. Moret et al, FED 2015
[4] E. Poli et al., CPC 225 (2018) 36-46 [5] M. Reich et al., FED 100 (2015) 73-80
[6] M. Weiland et al., 27th IAEA FEC (TH/6-3), 2018
[7] T. Blanken al, FED 2019

## Event detection example: Real-time plasma confinement state detector using Deep Learning

- Combines convolutional layers (CNN) + LSTM
- Based on [Matos, NF 2020]





## Model-based, dynamic state observer: merge model prediction and diagnostic measurements

- Amounts to performing a real-time simulation of the plasma time evolution, with corrections from measurements
  - Known in control literature as dynamic state observer, or Kalman filter.
  - Widely used in robotics, image processing, broad literature exists

e.g. [Kailath, Linear Estimation, Prentice Hall (2000)]



# Nonlinear observers: need linearization around nonlinear trajectory

Nonlinear model

$$egin{aligned} &x_{k+1} = f(x_k, u_k) \ &y_k = h(x_k) \end{aligned}$$

- A well-known observer for nonlinear systems is the Extended Kalman Filter (EKF).
  - Evolve state error covariance matrix S<sub>k</sub> together with state x<sub>k</sub>.
  - Matrices R<sub>k</sub> (sensor noise covariance) and Q<sub>k</sub> (process noise cov.) to be tuned

$$\begin{split} \hat{x}_{k|k} &= \hat{x}_{k|k-1} + L_k[y_k - h(\hat{x}_{k|k-1})] \quad \text{state meas. update} \\ \hat{x}_{k+1|k} &= f_k(\hat{x}_{k|k}, u_k) \quad \text{Predicted state} \\ L_k &= S_{k|k-1} H_k^T \Omega_k^{-1}, \Omega_k = H_k S_{k|k-1} H_k^T + R_k \quad \text{Kalman gain} \\ S_{k|k} &= (I - S_{k|k-1} H_k^T \Omega_k^{-1} H_k) S_{k|k-1} \quad \text{covariance meas. update} \\ S_{k+1|k} &= F_k S_{k|k} F_k^T + G_k Q_k G_k^T \quad \text{covariance time update} \end{split}$$

Other nonlinear filtering methods exist e.g. Unscented KF ...

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**Need Jacobians** 

 $egin{aligned} F_k &= rac{\partial f}{\partial x_k}\ G_k &= rac{\partial f}{\partial u_k}\ G_k &= rac{\partial h}{\partial h} \end{aligned}$ 

# 'Kalman gain' reflects confidence in models vs measurements

- If the system is linear, the Kalman Filter is the optimal filter
  - Gives the smallest state error covariance w.r.t. any other filter
- Though all systems are nonlinear, we can still use KF
- Useful features:
  - Model-based filter to remove diagnostic noise and obtain estimated states
    - (e.g. velocity from position measurements)
  - Can treat uncertainties in model parameters
    - Can represented as fictitious process noise entering the state equation
    - Can define *augmented* state = [plasma state; model parameters] to be estimated
  - Diagnostic fault detection
    - Single diagnostic residual increases  $\rightarrow$  detect fault  $\rightarrow$  eliminate from update law.
  - Anomaly detection
    - Sudden deviation of a set of diagnostic measurements from model → trigger disruption mitigation.

## Example: real-time density profile reconstruction on ASDEX-Upgrade using state observer

- RAPDENS model (related to RAPTOR)
  - Model combining 1D profile evolution and particle inventory model.
  - Update to these predictions using interferometer
     & bremsstrahlung measurements
  - Detection and rejection of diagnostic faults and model inaccuracies.
- Needs
  - Real-time capable simulator for 1D profiles
  - Ad-hoc models for transport coefficients + sources
  - Real-time diagnostics
- Future improvements of models, or diagnostics, feed into same state observer, no need to change controller.

[T. Bosman, Fus. Eng. Des, 2021] TCV implementation [F. Pastore, Poster Tuesday]



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#### **Details of 'Task'-based control layer**



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### Plasma state monitor translates continuous-valued plasma state estimate into discrete states

#### [T. Blanken NF 2019]



- Discrete representation of plasma state (including events)
  - Receives continuous-valued information from state reconstruction.
- User-configurable thresholds
  - Different thresholds for each tokamak.

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#### **Details of 'Task'-based control layer**



### Supervisor: map discrete-valued plasma state description into prioritized tasks

#### Rule-based mapping. Example:

	Plasma parameters are within defined 'normal' bounds	A 2/1 NTM is present (size = SMALL or MEDIUM)	A 2/1 NTM is present (size == LARGE)
Tasks (prioritized)	<ul> <li>2/1 NTM preemption</li> <li>β control</li> <li>q profile control</li> </ul>	<ul> <li>2/1 NTM stabilization</li> <li>β control with lower reference</li> </ul>	<ul> <li>Perform soft-stop (ramp-down)</li> </ul>
Control task parameters	<ul> <li>High β reference.</li> <li>2 MW EC on q=2.</li> </ul>	<ul> <li>Lower β reference.</li> <li>Increase EC power on q=2 until NTM is stabilized.</li> </ul>	<ul> <li>Appropriate soft-stop trajectory given present state.</li> <li>(OR trigger disruption mitigation etc)</li> </ul>

#### **Details of 'Task'-based control layer**


# Actuator manager decides in real-time which actuator resources are assigned to which control tasks



- Constrained optimization problem with both integer and continuous variables.
  - Heuristic approach works for case with few actuators / tasks.

**Example of RT actuator allocation for ITER control tasks** see [T. Vu et al, Fus. Eng Des 2019]

# Mixed-integer quadratic programming formulation of actuator allocation problems

[E. Maljaars & F. Felici, Fus. Eng Des 2017]

- Resource allocation problems have often been formulated in a flexible format as Mixed Integer (Quadratic) Programming problems
  - Optimization problem involves integer (and continuous) variables

```
\begin{array}{ll} \underset{X}{\text{minimize}} & J(x) = x^{\top} H x + f^{\top} x \\ \text{subject to} & A_{ineq} x \leq b_{ineq} \\ & x_{min} \leq x \leq x_{max} \\ & x_i \in \mathbb{N} \end{array}
```

- Cost function: things that are desired (easy to add/remove terms)
  - Actuator allocation: promote good / penalize bad allocations
- Constraints: things that must be satisfied (easy to add/remove terms)
  - For actuator allocation: actuator availability and allowed allocations

#### **Details of 'Task'-based control layer**



# Controllers execute (one or several) control tasks, receive resource allocations and send resource requests



- Generic interfaces for all controllers
- Enables use of resource-aware controllers (e.g. Model Predictive Control)
  - More details & examples in [Schuster, lecture Wednesday)

#### **Details of 'Task'-based control layer**



# Actuator interface translates generic actuator commands into (hardware-)specific commands for a given tokamak



### **Example of ITER EC actuator interface proposal**

See [G. Carannante proceedings EC-21 conference (2022)]

#### Function 1: Knowing where EC power is being deposited now



 NB Plasma information comes from plasma state reconstruction support functions

### **Example of ITER EC actuator interface proposal**

Function 2: Describe potential availability, now and in the future

PCS algos needing this info	Actuator interface Actuator availability in terms of P <sub>dep</sub> , rho <sub>dep</sub> , I <sub>ECCD</sub>	Translate availability in terms of k vector, power per mirror into rho, I_current drive	<b>Per launch point</b> , calculate: Availability of power, location, k & polarization vectors, (present and future)	Calculate set of potential states of EC system components (present and future)	Local readback of EC system component state + potentially settable states

 Needs representation of EC availability in terms of power/ polarization/angles of last mirror.

- Representation to be determined, likely a set of inequality constraints, or a tree
- Include mutual exclusion conditions etc

### **Example of ITER EC actuator interface proposal**

#### Function 3: 'Command' to inject EC at desired location

PCS control task prioritization	PCS command: "Deposit to given rho with given power and I <sub>cd</sub>	Decide X or O mode. Determine launch point, <b>k</b> vector, etc to achieve desired deposition. Find polarization vector <b>at</b> <b>mirror</b> for desired O/X mode.	Decide how to set launchers/gyrotrons/ switches/tl/polarizers to actuate command.	Positioning of mirror angles, polarizers, switches	Local control of EC system components
		(inv. ray tracing + optimization if multiple solutions)			

#### Separation of concerns:

- Actuator management on PCS side does optimization based only on effect of EC on plasma (+wall) in terms of rho, I<sub>ECCD</sub>, P<sub>absorbed</sub>, and decides desired EC system state at launch points.
- EC system decides how to actuate EC system components to obtain desired EC power at launch points.

#### Implementation aspects to promote algorithm portability

- Try to strictly separate parts of PCS software:
  - Tokamak-dependent / Tokamak-independent
  - PCS-dependent / PCS-independent



### **Outlook for supervisory control**

- Architectures are being tested successfully on various tokamaks
  - Also enable new experiments studying physics in better-controlled ways
- Solid, extensible architecture designed for ITER
- Tricks are in the details: implementing and validating:
  - State observers giving us all the physics quantities we need to know in real-time
  - Event detectors for all the N events we care about
  - Controllers for everything we want to control
    - Incl. resource-aware controllers, predictive controllers, ...
  - Program it all, validate and test it all
- From the control point of view, present research-oriented tokamaks are a dream
  - Many diagnostics, many flexible actuators -> 'pay' in control complexity
- What about a fusion reactor?
  - Run one scenario but fewer diagnostics and actuators

### Implementation challenges and software aspects

# A hierarchy of models is needed for different phases of controller design/validation/verification



#### Pre-shot model-based validation of discharge program ...

... & feedback of experimental data into model



#### Pre-shot model-based validation of discharge program ...

... & feedback of experimental data into model

- Operational limit checking:
  - Check that discharge program does not exceed operational boundaries (though we have real-time protection systems)
- Use best available "Flight Simulators" in closed-loop with a PCS (simulated or real)



#### Pre-shot model-based validation of discharge program ...

... & feedback of experimental data into model

- Operational limit checking:
  - Check that discharge program does not exceed operational boundaries (though we have real-time protection systems)
- Use best available "Flight Simulators" in closed-loop with a PCS (simulated or real)
- Deviations between pre-shot validation simulation and post-shot data contains valuable information
  - Improvement of models by changing device-specific parameters.
  - The physics we are trying to learn
  - Feed improved understanding into better models used for future control validations
  - Validated models (the code itself) are one of the key products of operating a tokamak



# Managing workflows of different stages of software validation is challenging but essential for future devices

- Validation of PCS software via closedloop simulations with plant models
- Verification & validation tests on:
  - Control software
  - Model software used to test the controls
- Need to do this:
  - Over ITER lifetime (several decades)
  - On several parallel versions of PCS software for various stages
  - While dozens++ of contributors propose changes and upgrades
- This is a "Large Software Project"
  - Need concepts from software engineering: continuous integration / deployment / DevOps



From [P. de Vries et al. Fus. Eng. Des 2018]

### **Continuous Integration (CI)**



- Automated, fast & frequent feedback of effects of code changes!
  - Requires codes with TESTS

### The importance of testing in software engineering

- Write tests together with code
  - For given input, expect a given output
  - As functionality expands, expand test suite
- Establish a 'contract', fixing expected code behaviour
- Run tests automatically and regularly





### **Types of tests**

- Various levels of testing:
  - Unit testing: test small functional units of code e.g. test an ODE solver
  - Integration tests: Tests of useful combinations of units
  - End-to-end tests: Test the whole thing
- Various aspects of Plasma Control software to be tested:
  - Functional tests of individual controllers (ITER: PCSSP)
  - Functional tests of combinations of controllers (ITER: PCSSP)
  - Tests that control code in simulation code same behaviour as code in production
    - PCSSP version vs RTF version (could be the same)
  - Hardware-in-the-loop tests
    - Tests of production PCS on real-time capable model of the whole system



#### The DevOps confusion

From: https://www.devops.ch/2017/05/10/devops-explained/





maximize **change** 

**Control algorithm developer** 



### The DevOps confusion



#### **Control algorithm developer**

**Tokamak operator** 



### The DevOps solution



- Dev: Automate (to the extent possible) all testing and deployment
  - Continuously test and deploy new software
- Ops: Provide platform for dev as close as possible to the real thing
  - The real-time control software environment + the models on which to test
- Run through this loop frequently

#### Promote frequent, rapid, small iterations





- Controllers and models are ultimately software projects
  - Transition from demonstrations or in-house tools to 'production' level codes
  - Role of open-source? -> leverage power of the community
- Software industry has developed methods for harnessing large collaborative software projects
  - Culture in fusion community has lagged behind, but is catching up
  - Promote this culture and educate ourselves on best practices / tools
- Essential role of software 'digital twins' for future tokamaks



T. Todd, in R. Dendy Plasma Physics p. 448 (1993)





#### **EPFL Graduate Course - Control & Operation of Tokamaks**

Next edition: February 2023 contact <u>federico.felici@epfl.ch</u> for more information

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F. Felici - Control Integration - ITER International School - San Diego July 2022



# Implementation of q profile + $\beta$ control on TCV including plasma state reconstruction.



#### 3 Tasks:

Task name	Activation	
Central co-CD	[0.4s-0.55s]	
2/1 NTM	[0.5s-2.5s]	
stabilization	+NTM presence	
eta control	[0.5s-2.5s]	

#### 2 Actuators:

Actuator name	Туре
EC launcher L4	co-CD (0.5MW)
EC launcher L6	co-CD (0.5MW)



#### For more details:

[T. Blanken Nucl. Fus. 2019] [T. Vu Fus. Eng. Des. 2019]





2/1 NTM onset (panel (f)), NTM stabilization takes priority 1, requests 0.5MW and gets L4 β control is activated as well, requests 1MW, but gets only the remaining L6 due to its lower priority








## TCV example: simultaneous NTM stabilization and $\beta$ control with real-time task prioritization

β control only, with both L4 and L6

6 NTM is detected and NTM stabilization takes priority 1





# Asynchronous response - intervene when threshold is exceeded

- Deviate from 'nominal' scenario to 'recover' the discharge
  - Should catch 'most' of remaining 1% cases
- Detect and track multiple events simultaneously
- Need to track various events:
  - Exceeding of limits related to proximity control
  - (N)TM presence / locked modes
  - Sawteeth, Minor disruptions
  - ELMs, Impurity influx
  - MARFE onset
  - (Real-time detectors needed for all these quantities..)
- Respond by targeted recovery actions, or ramp-down
- Leave as few cases as possible for DMS triggering



Repeated recovery of discharge based on MARFE position monitoring, acting on gas & heating [B. Sieglin, M. Maraschek, M. Bernert ASDEX Upgrade]

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**EURO***fusion* 

### **Outlook: towards resource-aware NTM control**

- First: Modified Rutherford Equation (MRE) model for w<sub>NTM</sub>(t)
  - Including empirical  $\Delta$ '(w) for TCV.
  - Reproduces island width evolution w(t) from w=0 to w=wsat
    - [M. Kong, NF 2019]
- Solving MRE in PCS resource-aware NTM controller
  - Estimate required power & deposition location for NTM preemption
  - Estimate required power for NTM suppression
  - Continuously update estimates **Plasma and Actuator State Reconstruction** based on plasma state RAPTOR MHD MRE **RT-diagnostics** observer analyses observer Realized actuator **RT Liuge** Plasma & actuator RAPTOR MRE commands states and limits predictor predictor Torbeam User parameters Sawtooth Density detector observer **External Library Blocks** Activation Power w(t) Assigned resources evaluation evolution Controller requests (range of power,  $\rho$ , CD) NTM Plasma & actuator controller Core MRE solver states Controller commands dw (power,  $\rho$ , CD)  $\frac{dt}{dt} = f(w,t)$ Controller parameters per task

### **Outlook: towards resource-aware NTM control**

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  - Estimate required power for NTM suppression



# Simulation of real-time MRE-based control of NTMs: continuously predict w<sub>ntm</sub>(t) evolution

#### • TCV experiment:

- Sweep 800kW EC beam across q=2 surface.
- NTM stabilized when  $\rho_{\text{dep}} \, \text{crosses} \, \rho_{\text{q=2}}$

#### Simulation using MRE model:

- Predict w(t) time evolution for different EC power levels.
- Predicts NTM stabilization at expected time for this power level.
- Predicts that lower power would not have stabilized the mode.

